

**4731.0423 DETERMINATION OF A<sub>1</sub> AND A<sub>2</sub>.**

Subpart 1. **Generally.** Values of A<sub>1</sub> and A<sub>2</sub> for individual radionuclides, which are the bases for many activity limits elsewhere in this chapter, are given in part 4731.0422, subpart 1a. The curie (Ci) values specified are obtained by converting from the Terabecquerel (TBq) values. The Terabecquerel values are the regulatory standard. The curie values are for information only and are not intended to be the regulatory standard. Where values of A<sub>1</sub> and A<sub>2</sub> are unlimited, it is for radiation control purposes only. For nuclear criticality safety, some materials are subject to controls placed on fissile material.

Subp. 2. **Individual radionuclides; not listed in part 4731.0422, subpart 1a.** For individual radionuclides whose identities are known, but which are not listed in part 4731.0422, subpart 1a, the A<sub>1</sub> and A<sub>2</sub> values contained in subpart 6 may be used. Otherwise, the licensee shall obtain prior commissioner, NRC, or agreement state approval of the radionuclides not listed in part 4731.0422, subpart 1a, before shipping the material.

Subp. 2a. **Individual radionuclides; not listed in part 4731.0422, subpart 3.** For individual radionuclides whose identities are known, but which are not listed in part 4731.0422, subpart 3, the exempt material activity concentration and exempt consignment activity values contained in subpart 6 may be used. Otherwise, the licensee shall obtain prior commissioner, NRC, or agreement state approval of the exempt material activity concentration and exempt consignment activity values for radionuclides not listed in part 4731.0422, subpart 3, before shipping the material.

Subp. 2b. **Prior approval.** The licensee must submit requests for prior approval, described under subparts 2 and 2a, to the commissioner, NRC, or agreement state, according to this chapter.

Subp. 3. **Radioactive decay chain.** In the calculations of A<sub>1</sub> and A<sub>2</sub> for a radionuclide not in part 4731.0422, subpart 1a, a single radioactive decay chain, in which radionuclides are present in their naturally occurring proportions and in which no daughter nuclide has a half-life longer than ten days or longer than that of the parent nuclide, shall be considered as a single radionuclide. The activity to be taken into account and the A<sub>1</sub> and A<sub>2</sub> value to be applied shall be those corresponding to the parent nuclide of the chain. In the case of radioactive decay chains in which any daughter nuclide has a half-life longer than ten days or greater than that of the parent radionuclide, the parent and those daughter radionuclides shall be considered as mixtures of different radionuclides.

Subp. 4. **Radionuclide mixture.** For mixtures of radionuclides whose identities and respective activities are known, the following conditions apply:

A. For special form radioactive material, the maximum quantity transported in a Type A package:

$$\sum_i \frac{B(i)}{A_1(i)} \leq 1$$

where B(i) is the activity of radionuclide i in special form and A<sub>1</sub>(i) is the A<sub>1</sub> value for radionuclide i.

B. For normal form radioactive material, the maximum quantity transported in a Type A package:

$$\sum_i \frac{B(i)}{A_2(i)} \leq 1$$

where  $B(i)$  is the activity of radionuclide  $i$  in normal form and  $A_2(i)$  is the  $A_2$  value for radionuclide  $i$ .

C. If the package contains both a special and normal form radioactive material, the activity that may be transported in a Type A package:

$$\sum_i \frac{B(i)}{A_1(i)} + \sum_j \frac{C(j)}{A_2(j)} \leq 1$$

where  $B(i)$  is the activity of radionuclide  $i$  in special form,  $A_1(i)$  is the  $A_1$  value for radionuclide  $i$ ,  $C(j)$  is the activity of radionuclide  $j$  in normal form, and  $A_2(j)$  is the  $A_2$  value for radionuclide  $j$ .

D. Alternatively, an  $A_1$  value for mixtures of special form material may be determined as follows:

$$A_1 \text{ for mixture} = \frac{1}{\sum_i \frac{f(i)}{A_1(i)}}$$

where  $f(i)$  is the fraction of activity of radionuclide  $i$  in the mixture and  $A_1(i)$  is the appropriate  $A_1$  value for radionuclide  $i$ .

E. Alternatively, the  $A_2$  value for mixtures of normal form material may be determined as follows:

$$A_2 \text{ for mixture} = \frac{1}{\sum_i \frac{f(i)}{A_2(i)}}$$

where  $f(i)$  is the fraction of activity of radionuclide  $i$  in the mixture and  $A_2(i)$  is the appropriate  $A_2$  value for radionuclide  $i$ .

F. The exempt activity concentration for mixtures of radionuclides may be determined as follows:

$$\text{Exempt activity concentration for mixture} = \frac{1}{\sum_i \frac{f(i)}{[A](i)}}$$

where  $f(i)$  is the fraction of activity concentration of radionuclide  $i$  in the mixture, and  $[A](i)$  is the activity concentration for exempt material containing radionuclide  $i$ .

G. The activity limit for an exempt consignment for mixtures of radionuclides may be determined as follows:

$$\text{Exempt consignment activity limit for mixture} = \frac{1}{\sum_i \frac{f(i)}{A(i)}}$$

where  $f(i)$  is the fraction of activity of radionuclide  $i$  in the mixture, and  $A(i)$  is the activity limit for exempt consignments for radionuclide  $i$ .

**Subp. 5. Activities unknown.**

A. When the identity of each radionuclide is known, but the individual activities of some of the radionuclides are not known, the radionuclides may be grouped and the lowest  $A_1$  or  $A_2$  value, as appropriate, for the radionuclides in each group may be used in applying the formulas in subpart 4. Groups may be based on the total alpha activity and the total beta/gamma activity when these are known, using the lowest  $A_1$  or  $A_2$  values for the alpha emitters and beta/gamma emitters.

B. When the identity of each radionuclide is known but the individual activities of some of the radionuclides are not known, the radionuclides may be grouped and the lowest  $[A]$  (activity concentration for exempt material) or  $A$  (activity limit for exempt consignment) value, as appropriate, for the radionuclides in each group may be used in applying the formulas in subpart 4. Groups may be based on the total alpha activity and the total beta/gamma activity when these are known, using the lowest  $[A]$  or  $A$  values for the alpha emitters and beta/gamma emitters, respectively.

**Subp. 6. General values for  $A_1$  and  $A_2$ .**

	$A_1$		$A_2$	
Contents	(TBq)	(Ci)	(TBq)	(Ci)
Only beta- or gamma-emitting radionuclides are known to be present	$1 \times 10^{-1}$	2.7	$2 \times 10^{-2}$	$5.4 \times 10^{-1}$
Alpha-emitting nuclides, but no neutron emitters are known to be present <sup>a</sup>	$2 \times 10^{-1}$	5.4	$9 \times 10^{-5}$	$2.4 \times 10^{-3}$
Neutron-emitting nuclides are known to be present or no relevant data are available	$1 \times 10^{-3}$	$2.7 \times 10^{-2}$	$9 \times 10^{-5}$	$2.4 \times 10^{-3}$

<sup>a</sup> If beta- or gamma-emitting nuclides are known to be present, the A<sub>1</sub> value of 0.1TBq (2.7 Ci) should be used.

Contents	Activity concentration for exempt material (Bq/g)	Activity concentration for exempt material (Ci/g)	Activity limits for consignments (Bq)	Activity limits for consignments (Ci)
Only beta- or gamma-emitting radionuclides are known to be present	$1 \times 10^1$	$2.7 \times 10^{-10}$	$1 \times 10^4$	$2.7 \times 10^{-7}$
Alpha-emitting nuclides, but no neutron emitters are known to be present	$1 \times 10^{-1}$	$2.7 \times 10^{-12}$	$1 \times 10^3$	$2.7 \times 10^{-8}$
Neutron-emitting nuclides are known to be present or no relevant data are available	$1 \times 10^{-1}$	$2.7 \times 10^{-12}$	$1 \times 10^3$	$2.7 \times 10^{-8}$

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